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Strongly Reduced Penetration of Atomic Deuterium in Radiation-Damaged Tungsten

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Radiation-damaged tungsten is exposed to high-flux, low-energy deuterium plasmas at self-bias conditions. We observe that the fraction of deuterium that penetrates is only $10^{-5}$–$10^{-7}$ of the plasma flux and strongly dependent on the local surface temperature. We propose that deuterium does not directly penetrate bulk tungsten but that it thermalizes at the surface, where it forms a protective chemisorbed layer. We find an energy barrier of 1–2 eV between the surface and bulk, causing the influx of deuterium to be low as compared to the number of defects and leading to slow filling of the damaged layer.

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Tungsten is foreseen as the main candidate material for use in the divertor of the experimental fusion reactor ITER. The divertor serves as the exhaust of the tokamak where the helium ash is removed. It is particularly in this region where strong interaction of plasma with plasma-facing components (PFCs) takes place. The plasma in the divertor region has a high electron density ($10^{20}$–$10^{21}$ m$^{-3}$) and a temperature of 1–10 eV. This leads to extremely intense particle and energy flux densities of \(~10^{24}\) ions m$^{-2}$ s$^{-1}$ and \(~10\) MW m$^{-2}$, respectively [1]. Its high thermal conductivity and melting point as well as its low erosion rate make tungsten favorable over many other materials. Another essential property of PFCs is that the uptake of tritium should be limited. For safety and efficiency reasons, the total tritium inventory in ITER should be kept below 700 g [2]. For tungsten it is known that the retention of hydrogen particles is low (<$10^{-4}$ atomic fraction). However, radiation damage in a fusion reactor can lead to enhancement of the retention by orders of magnitude [3,4].

In this Letter, we present deuterium retention studies for radiation-damaged tungsten targets, exposed to atomic deuterium plasmas under ITER-like conditions. For ion temperatures of a few eV, it is known that 90%–95% of the deuterium is directly reflected [5,6]. However, the plasma flux is so high that there are large amounts of remaining deuterium atoms that interact with the tungsten surface. This results in a very complex plasma-surface interaction leading to adsorption of deuterium at the surface, recombination of atomic deuterium to molecules and release into the vacuum and/or back into the plasma [7,8], penetration of deuterium in tungsten, etc. The situation is very schematically depicted in Fig. 1 with $\Phi_{\text{plasma}}$ the plasma flux and $\Phi_{\text{back}}$ the back flux containing directly reflected deuterium as well as thermalized deuterium leaving the surface as atoms and/or molecules. The effective ingoing flux $\Phi_{\text{in}}$ is the fraction $f$ of the plasma flux that enters the tungsten material. In the present work we have monitored the diffusion of deuterium inwards, from which we were able to directly determine $\Phi_{\text{in}}$ and thereby the fraction $f$. We obtain a value for $f$ that is very small: $10^{-5}$–$10^{-7}$. This fraction is strongly dependent on the local surface temperature and orders of magnitude lower than expected on the basis of the direct reflection mentioned above. We propose that this small value for $f$ originates in the deuterium not directly entering bulk tungsten but thermalizing and neutralizing at the surface forming a protective chemisorbed layer. Before diffusing into the bulk, the atoms need to cross an energy barrier that strongly limits the influx. We will show that an activation energy for crossing the barrier of 1–2 eV is consistent with the temperature dependence of the influx.

Polycrystalline tungsten targets (PLANSEE, 99.96% purity, Ø 20 mm, 1 mm thick) were mechanically polished until mirror finished and subsequently heated for 1 h at 1273 K at a background pressure of $5 \times 10^{-4}$ Pa. The targets were preirradiated at room temperature with 12.3 MeV $W^{4+}$ ions at the 3 MV ion accelerator at IPP Garching to homogeneous lateral damage. The fluence of the preirradiations was taken such that a damage level of 0.45 displacements per atom was reached (average displacement energy 90 eV). We have previously shown that this fluence is sufficient to saturate the damage level and that the damage extends to a depth of about 1.5 μm below the surface [3]. The surface cleanliness was checked by x-ray photoelectron spectroscopy. Since the targets were

FIG. 1. The influx $\Phi_{\text{in}}$ is only a small fraction $f$ of the incoming plasma flux $\Phi_{\text{plasma}}$. 

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stored under ambient conditions, they were covered with nanometer-thick native layers of oxygen and carbon \[3,9\]. During plasma exposure, most of these layers are removed within the first few seconds, although the presence of in particular some oxygen cannot be excluded \[10\]. After plasma exposure, small amounts of Mo, Ca, and F were found to be present on the surface. In most cases, the concentrations were well below 10 at.%. These impurities may slightly affect the protective layer and the energy barrier that we found in our measurements.

The predamaged tungsten targets were exposed to intense deuterium plasma beams at the linear plasma generator Pilot-PSI \[11\]. The plasma was created by a cascaded arc source and transported to the target by an axial magnetic field of 0.4 T. During the exposures, the targets were water cooled and electrically floating. Electron density and temperature profiles of the plasma beam were determined by Thomson scattering \[12\]. The plasma beam has approximately a Gaussian profile with a maximum electron density and temperature of about \(10^{20}\) m\(^{-3}\) and 1 eV, respectively. The ion flux on the targets can be estimated by applying the Bohm criterion \[13\] and ranges from \(2 \times 10^{24}\) ions m\(^{-2}\) s\(^{-1}\) in the center of the target to \(5 \times 10^{23}\) ions m\(^{-2}\) s\(^{-1}\) at the edges. For our experimental conditions, the deuterium in the plasma predominately consists of atomic ions \[14,15\]. The kinetic energy of the ions arriving at the surface can be estimated by considering a drifting Maxwellian velocity distribution for the ions in the plasma and an acceleration over the plasma sheath towards the surface. Assuming equal electron and ion temperatures in the plasma \[15\], this leads to an ion kinetic energy of roughly 5 eV. The temperature profile of the tungsten surface during exposure was measured with a fast infrared camera (FLIR SC7500-MB) and varies between 560 K in the center of the target to 470 K at the edges (the emissivity of the tungsten targets used was 0.07, a value verified \textit{ex situ} with the help of a thermocouple measurement). The absolute uncertainty in the temperatures is of the order of 25 K and is mainly caused by the uncertainty in the emissivity and the transmission of the infrared radiation through the vacuum window. The relative uncertainty in the temperatures is a few K.

Deuterium depth profiles were measured one month after plasma exposure by nuclear reaction analysis (NRA) making use of the nuclear reaction \(\text{D}(^{3}\text{He}, p)^{3}\text{He}\) \[16\]. The \(^{3}\text{He}\) beam spot of about 1 mm in diameter was positioned at six spots on the target: two close to the center and two at 3 mm and two at 6 mm from the center at opposite sides. In this way, we obtain deuterium depth profiles for three different exposure temperatures, i.e., 560, 530, and 500 K. Differences between the measurements taken at the same temperature were small, and only averages will be shown. At each position, the beam energy was scanned from 690 keV to 4.0 MeV to determine the deuterium concentration with a resolution of about 0.5 \(\mu\text{m}\) down to a depth of 6 \(\mu\text{m}\). The depth profiles of retained deuterium were calculated from the measured proton energy distributions with the NRA-DC program \[17\]. The total amount of deuterium in the top 6 \(\mu\text{m}\) was obtained by integrating the depth profiles.

Five identical tungsten targets were exposed to deuterium plasma. The plasma fluxes and the corresponding surface temperatures were very similar in all experiments. The difference between the targets was the exposure time that was varied over almost 2 orders of magnitude, from 30 to 2250 s. This means a variation in fluence in the center of the targets from about \(6 \times 10^{25}\) to \(4.5 \times 10^{27}\) m\(^{-2}\) deuterium ions (fluence equals time integrated flux).

The NRA results for three of the targets are shown in Figs. 2(a)–2(c). For an exposure time of 75 s [Fig. 2(a)], the deuterium is retained mostly within 0.5 \(\mu\text{m}\) from the surface for all three temperatures. At longer exposure times [Fig. 2(b)], it is clear that, in particular for the highest temperature, deuterium penetrates deeper in the material. This penetration continues for the longest exposure time as shown in Fig. 2(c). The observations qualitatively agree with a process that is determined by diffusion: deeper penetration of deuterium for longer exposures and higher

![FIG. 2](color online). Deuterium depth distributions in damaged (0.45 dpa) tungsten targets exposed to high-flux deuterium plasmas at self-bias conditions (a)–(c) and biased at \(-40\) V (d). The exposure times are as indicated. The deuterium depth profiles were measured at three positions on the target (center at 560 K, squares; 3 mm off-center at 530 K, circles; 6 mm off-center at 500 K, triangles).
The experimentally observed durations for complete saturation of the damaged layer with deuterium are long compared to what would be expected on the basis of the large plasma flux and the diffusion of deuterium in tungsten. Using the Frauenfelder diffusion coefficient of deuterium in tungsten of \( D_F = D_0 \exp(-0.39/(k_BT)) \) with \( D_0 = 2.9 \times 10^{-7}\text{ m}^2\text{s}^{-1} \) [18] shows that for temperatures of 500–560 K deuterium diffuses in less than a second through the damaged layer. The damaged layer has an areal density of \( \sim 10^{21}\text{ m}^{-2} \). The plasma flux, after correction for the direct reflection of deuterium from tungsten, amounts to \( 10^{23}\text{ m}^{-2}\text{s}^{-1} \). This means that in 1 s plasma exposure there is ample deuterium available to fill the traps. This simple estimate thus yields a time scale that is more than 3 orders of magnitude faster than the experimentally observed saturation times. Uncertainties in the Frauenfelder diffusion coefficient [19] cannot explain this.

Why then is the experimentally observed filling of traps with deuterium 3 orders of magnitude slower than expected on the basis of plasma flux and diffusion? The assumption in the above considerations is that the high-flux plasma, after correction for reflection, directly penetrates into the outermost layers of the target. This seems to be a realistic assumption, since the introduction of atomic deuterium in tungsten is known to be exothermic [20] and since the incoming ions in addition have a kinetic energy of 5 eV. However, our results strongly suggest that the assumption is not valid. We propose that, instead of directly penetrating tungsten, the deuterium ions thermalize and neutralize at the surface, where they form a chemisorbed layer. The presence of this layer in fact plays an important role in the thermalization process: Deuterium-deuterium collisions cause energy pooling, thereby decreasing the energy of the incoming ions preventing direct penetration. This interpretation is supported by other experiments, where it was shown that a chemisorbed deuterium layer on a barium surface actively decreases the interaction between incoming deuterium and barium atoms [21]. The contribution of the chemisorbed layer to the thermalization of deuterium means that it “protects” tungsten from direct penetration by deuterium. To diffuse from this protective chemisorbed layer at the surface into bulk tungsten, deuterium needs to be activated over an energy barrier of a few eV [20,22]. This energy barrier reduces the effective influx of deuterium by many orders of magnitude to values so low that the filling of the damaged layer with deuterium is limited by the amount of traps and not by the diffusion coefficient. We will show that this interpretation leads to a consistent picture explaining the filling of the damaged layer being orders of magnitude slower than initially expected.

To determine values for \( \Phi_{\text{in}} \) that are consistent with our experimental observations, we carried out TMAP7 simulations. TMAP7 is a one-dimensional program that solves the diffusion equation for deuterium in materials [23]. The program includes trapping of deuterium and recombination to molecules at the surface. Following our previous results [9], we assume three different trap sites for deuterium in tungsten with trap densities of \( 2 \times 10^{-1}, 4.5 \times 10^{-3}, \) and \( 6.5 \times 10^{-1}\) at. % for trapping energies of 1.2, 1.4, and 1.85 eV, respectively. For the diffusion coefficients of deuterium in tungsten, the above-mentioned Frauenfelder values were used. For recombination of two atoms at the surface to a molecule, a coefficient of \( 3.2 \times 10^{-15}\text{ m}^4\text{s}^{-1}\exp(-1.16/(k_BT)) \) was taken [24]. The simulations were run with the flux varied until good agreement with the experiments was obtained. The results of the simulations are shown in Fig. 3. The deuterium influxes \( \Phi_{\text{in}} \) used were \( 2 \times 10^{19} \) (560 K), \( 1.8 \times 10^{18} \) (530 K), and \( 2.5 \times 10^{17}\text{ m}^{-2}\text{s}^{-1} \) (500 K), which means that \( f \) is in the range of \( 10^{-5}–10^{-7} \). Note that \( \Phi_{\text{in}} \) is the deuterium influx in the first nanometer of the material. Part of this influx leaves the surface again via recombination. To fit the experimental data, the simulated curves have slightly been shifted upwards by \( 1 \times 10^{20}\text{ m}^{-2} \). The kinks in the simulated curves for the two highest temperatures at 750
local temperature of the surface during plasma exposure.

the trapping of deuterium per unit time was the limiting

erable amounts of defects [26]. Also in these experiments

the diffusion of deuterium in tungsten containing consid-

extent reminiscent of what has been observed by others on

fast filling of the damaged layer. Our results are to some

the bulk, leading to high concentrations of deuterium and

This suggests that, upon their acceleration towards the

influx of deuterium, 

of the surface with its protective layer. For a target biased at

as compared to the plasma flux. Obviously, at self-bias

effective influx of deuterium by many orders of magnitude

surface modifications at self-bias conditions can readily be understood in terms of the

absence of direct penetration, as was shown in the present

The surface modifications can be related to an oversatura-
tion of tungsten with deuterium originating in the direct

penetration. The absence of surface modifications at self-
bias conditions can readily be understood in terms of the

absence of direct penetration, as was shown in the present

Letter.

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